

Optimization of Quasi-helically Symmetric Equilibria

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- 1 Stellarator Optimization
- 2 Optimization for energetic particle transport
- 3 Improved coil algorithms
- 4 Non-resonant divertors for QS
- 5 New QHS configuration for midscale experiments

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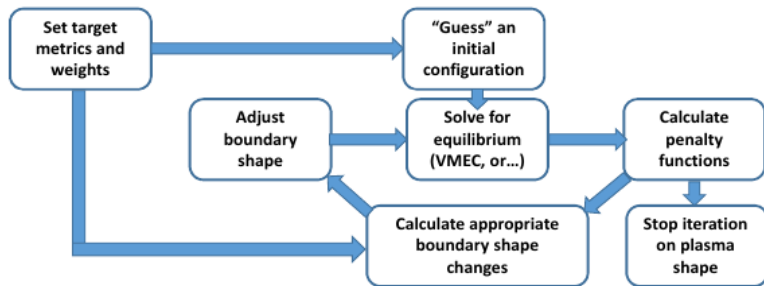
Stellarators offer a reliable reactor concept with low recirculating power

- Advantages of the stellarator concept
 - Do not rely on current: **No current driven disruptions**
 - Do not require current drive: **Low recirculating power**
 - Not subject to Greenwald density limits: **High density operation possible**
- Difficulties of the stellarator concept
 - More complicated design: **Increased cost of construction.** Opportunity for **advanced manufacturing to reduce costs**
 - **Lack of experimental data.** Opportunities for mid-scale devices to significantly **advance the concept**

Optimization procedures can find improved stellarator configs

Define a boundary: $R = \sum_{m,n} R_{m,n} \cos(m\theta - n\zeta)$, $Z = \sum_{m,n} Z_{m,n} \sin(m\theta - n\zeta)$

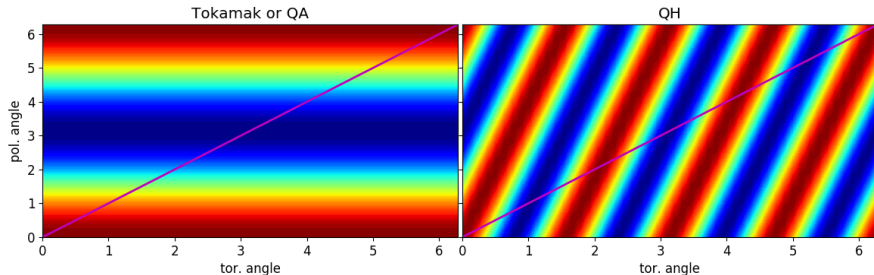
- Define targets to optimize and set weights for targets
- Solve for equilibrium, evaluate target functions
- Perturb $\mathbf{R}$, $\mathbf{Z}$ in an optimization scheme



New understanding allows optimization beyond HSX, W7-X

- Already demonstrated optimization for neo-classical transport, quasisymmetry, low plasma currents
- Breakthroughs in recent years allow optimization for new phenomena, and new improvements
 - Turbulent transport ([B. Faber - next talk](#))
 - Energetic particle confinement
 - Simplified coils ([see posters by L. Singh P73 and T. Kruger P72 Thur.](#))
 - Non-resonant divertors ([see poster by H. Frerichs P80 Thur.](#))
 - MHD stability (not in this talk, [see poster by J. Schmitt P30 Tues.](#))

Quasisymmetry - Tokamak transport in stellarator configurations

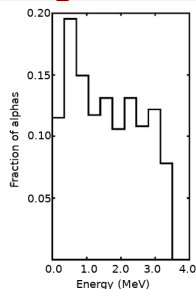


Benifits of QH

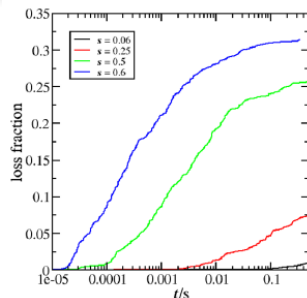
- Low flow damping in direction of symmetry (Gerhardt PRL 2005)
- Short connection lengths, lower bootstrap currents, lower Shafranov shift (than QA).

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Alpha particle losses may be a driving factor for stellarator reactor design



- ARIES-CS predicts 5% Alpha Energy loss
 - ARIES-CS has volume 450 m^3 and $B_0 = 5.6 \text{ T}$
 - Lost particles impact specific areas - above heat flux limits
- Henneberg shows new QA with particle loss $\approx 6\%$ loss at mid-radius
 - Volume of 1900 m^3 at $B_0 = 5 \text{ T}$
 - Loss suppressed deep in core



- Lotz (1992) predicts that QH and stellarators without bootstrap current should have better confinement
 - QH config. had 3% loss at Aspect Ratio 20

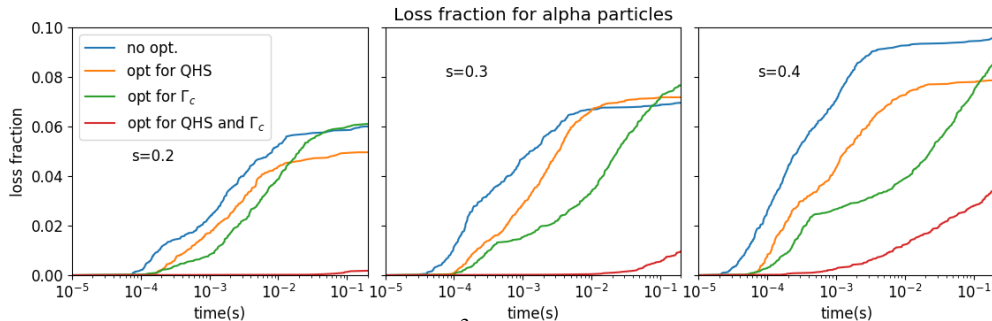
Mau FST 2008, Henneberg NF 2019, Lotz PPCF 1992

Γ_c metric aligns $J_{\parallel}$ contours with flux surfaces

- $J = \oint v_{\parallel} ds; \left\langle \frac{d\psi}{dt} \right\rangle = \frac{1}{Ze\tau_b} \frac{\partial J}{\partial \theta}; \left\langle \frac{d\theta}{dt} \right\rangle = -\frac{1}{Ze\tau_b} \frac{\partial J}{\partial \psi}$
- If $J = J(\psi)$ then $\dot{\psi} = 0$ and the particle does not drift off a flux surface
- $\Gamma_c \sim \sum_{E/\mu} \sum_{\text{wells}} \int_b \arctan^2 \left(\langle \dot{\psi} \rangle / \langle \dot{\theta} \rangle \right) \tau_b$
- Γ_c is related to the ratio of the average radial drift, to the average poloidal drift; i.e. if $\Gamma_c = 0, J = J(\psi)$
- Minimizing Γ_c should improve energetic particle confinement
- Testing effectiveness of Γ_c
 - Start with a QHS configuration and attempt to optimize QHS, and Γ_c , separately and together
 - Evaluate equilibria with collisionless Monte Carlo calculation that mimic alpha particle production

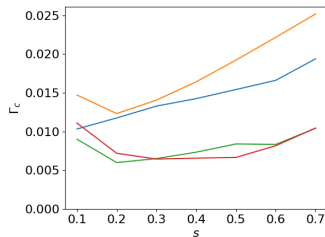
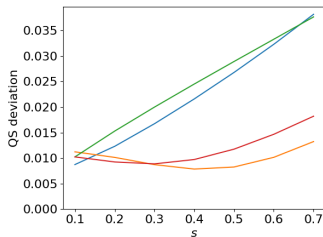
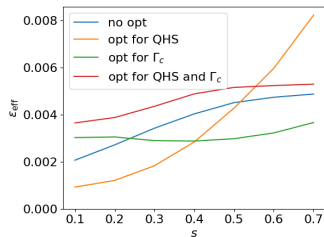
Nemov PoP 2008

Optimizing for Γ_c and QHS reduces collisionless losses to reactor relevant levels



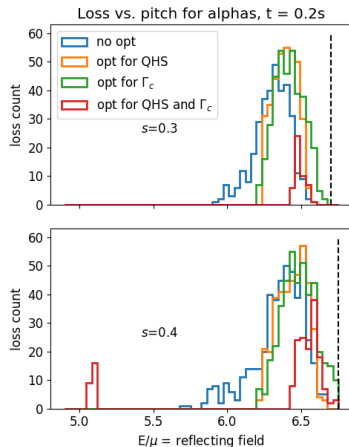
- All configurations scaled to 450 m^3 and $B_0 = 5.6 \text{ T}$ at aspect ratio 6.7
- Prompt losses entirely eliminated in best performing case (red), and below 1% within $s=0.3$

Improvements in alpha confinement is not correlated to ϵ_{eff}



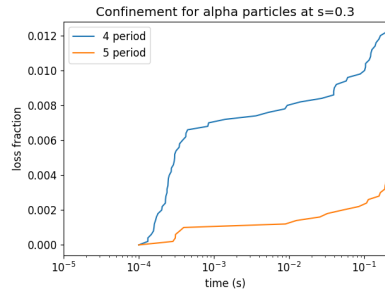
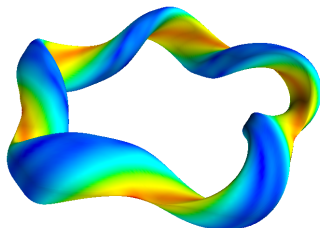
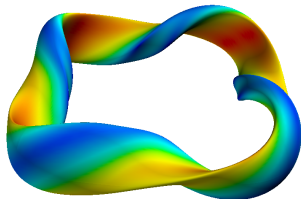
- Optimizer succeeds in optimizing QHS and Γ_c
- Improvement of alpha confinement despite worse ϵ_{eff} across most of the radius

Loss reduction appears mostly at trapped passing boundary



- Most problematic region is near the trapped passing boundary
- The best confinement case (red) sacrifices confinement of deeply trapped particles to better confine particles near the trapped passing boundary
- If $p = p(\psi)$ and alpha velocity is isotropic, then fewer particles will be born deeply trapped than at the trapped passing boundary

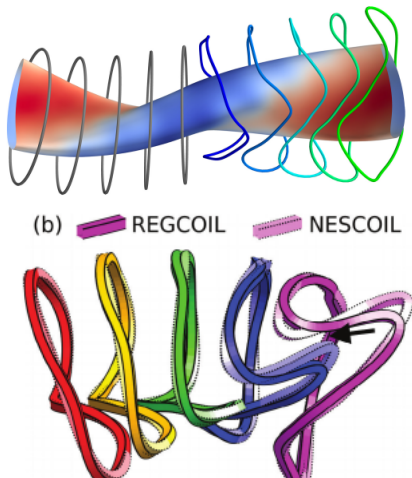
Five period configuration: losses below 1% at $s=0.3$



- Both configurations at aspect ratio 6.7
- Collisionless alpha losses below any published result to date

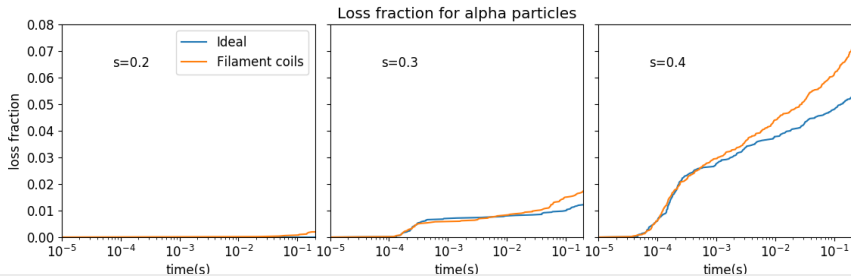
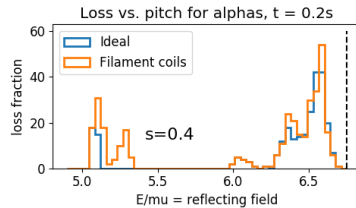
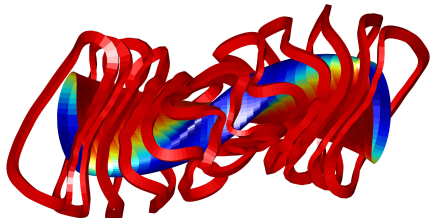
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New codes have advanced coil design

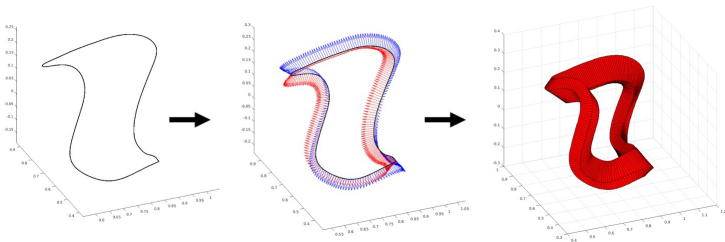
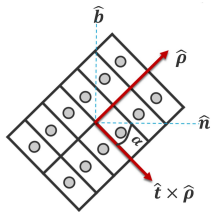


- FOCUS: solve coils in 3D space. (Zhu NF 2017)
- REGCOIL: solve for current potential on boundary surface, coils are contours of current potential. (Landreman NF 2017)
- Additional codes (not shown) ONSET, COILOPT++

FOCUS + REGCOIL accurately reproduce boundary

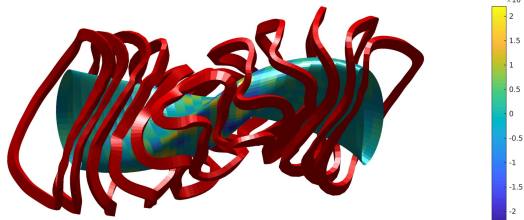


Developed new optimization for finite build coils



- Specify angle α , orientation of coil winding pack
- Optimize over α as additional parameter along with coil position

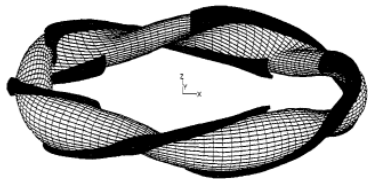
See L. Singh P73 Thur.



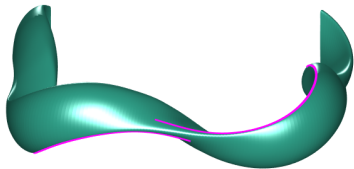
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Quasi-symmetric stellarators may not be compatible with helical or island divertors

Early W7-X



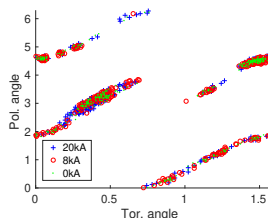
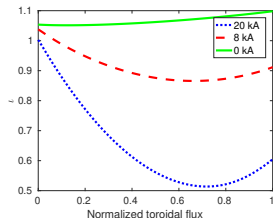
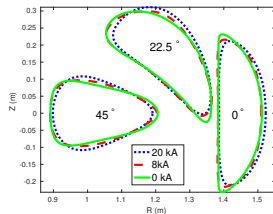
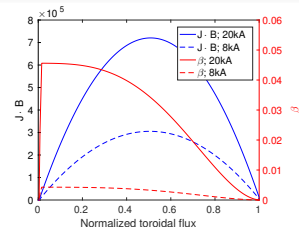
HSX



- Island divertor and helical divertors may not be suitable for other optimizations with finite bootstrap current, such as quasi-symmetric stellarators
- Early designs of W7-X exploited non-resonant divertors
- Stellarator shapes tend to have “ridges” on the surface
- Field lines follow along the ridges, like in a tokamak X-point
- **Difference from tokamaks: finite toroidal extent**

Strumberger NF 1992, Bader PoP 2017

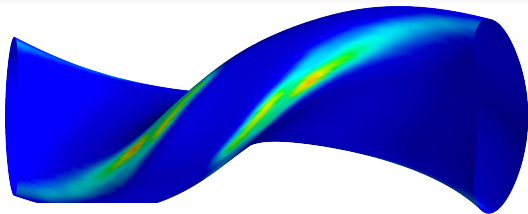
Non-resonant divertors resilient to plasma evolution



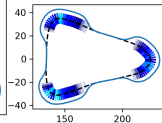
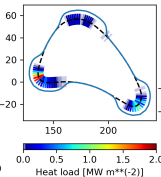
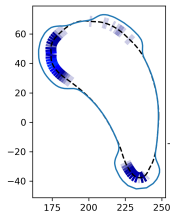
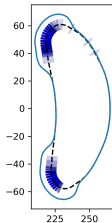
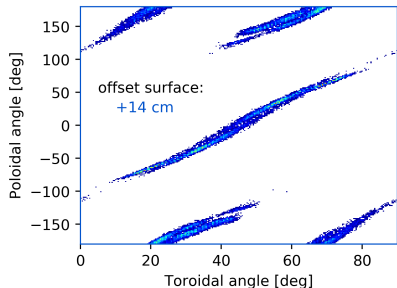
- Quasi-symmetric stellarators will have finite bootstrap currents and Shafranov shifts
- Rotational transform profiles will alter making island divertors difficult
- Non-resonant divertors promise similar performance despite changes in plasma boundary shape

Bader PoP 2017

Construction of a non-resonant divertor

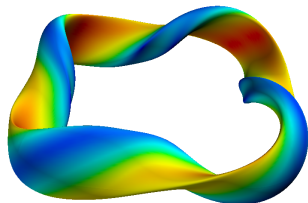


- Calculate strike patterns on uniform wall
- Create troughs in strike locations
- Iterate until uniform distribution
- Check resiliency
- See H. Frerichs poster P.80 Thur.



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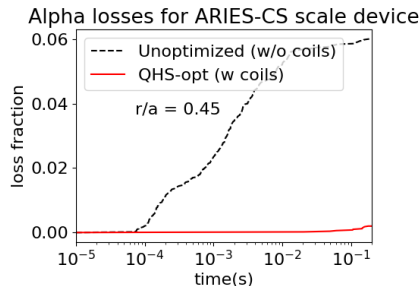
Midscale experiment can advance QS knowledge



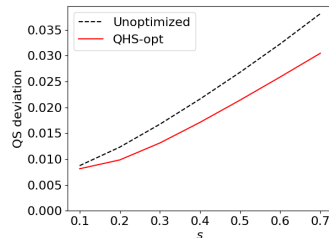
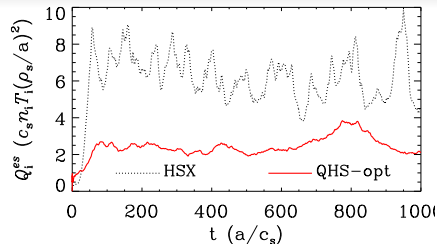
- Phased approach begins at 1.25 T, upgrade to 2.5 T
- Physics goals: Control turbulent transport, demonstrate good EP confinement, validate non-resonant divertor concept

Param.	Initial	Upgrade
$R(\text{m})$	2.0	2.0
$a(\text{m})$	0.3	0.3
$B(\text{T})$	1.25	2.5
ECH (MW)	0.5	1.0
NBI (MW)	0.0	1.0
$n (10^{20} \text{ m}^{-3})$	0.15	0.3
$T_e (\text{keV})$	3.2	4.0
$T_i (\text{keV})$	0.3	4.0
ν_i^*	0.4	0.005
$\tau_{ei} (\text{s})$	0.5	0.9
$\tau_E (\text{s})$	0.06	0.07

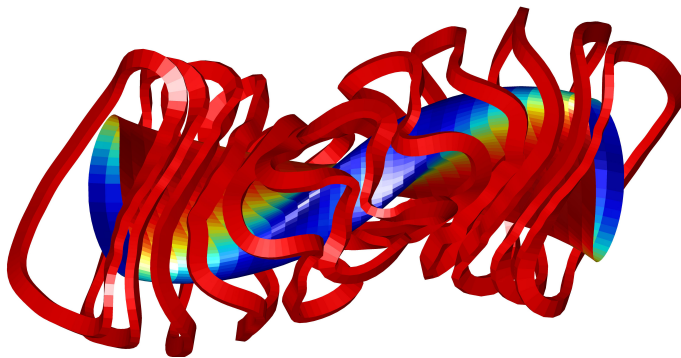
Excellent EP confinement with reduced turbulent transport



- EP losses almost entirely eliminated at $s=0.2$
- Turbulent heat flux reduced by factor of ≈ 3
- Slightly improved QHS metric

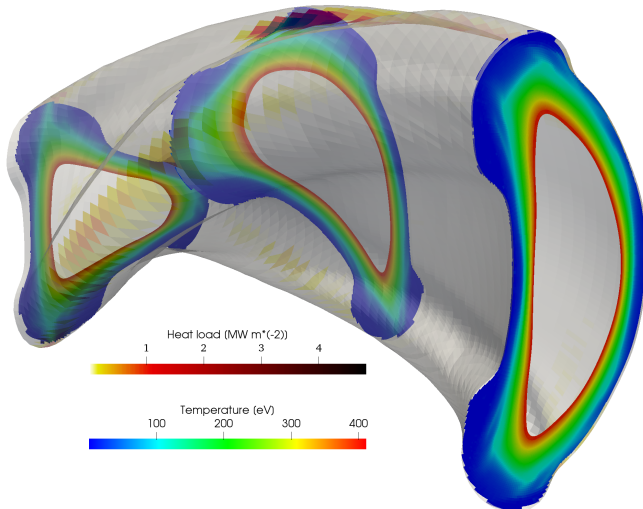


New coil design capabilities improve fidelity of magnetic structure



Min. Coil-plas. dist. (Single filament)	19.5 cm
Ave. Coil-plas. dist. (Single filament)	22.5 cm
RMS error (Single filament)	0.53%
Min. Coil-plas. dist. (Multi-filament)	14.6 cm
Ave. Coil-plas. dist. (Multi-filament)	17.7 cm
RMS error (Multi-filament)	0.60%

Expanded wall to test resonant divertor



- First attempt at generating walls for non-resonant divertors
- Localized toroidal hotspots indicate some finessing is necessary
- Iteration to improve divertor has begun
- See [H. Frerichs poster P.80 Thur](#) for more details

New metrics, tools, and optimizers improve design for next generation of optimized equilibria

- Energetic particle confinement at Tokamak levels within the mid-radius
- New coil algorithms can produce high fidelity, realistic coils
- First explorations of non-resonant divertor design has begun
- Stay tuned for next talk by Benjamin Faber on turbulent transport optimization